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### Group-averaged neutron and gammaray cross sections for radiation protection and shielding calculations for nuclear reactors

effici.
oprotect. Sections efficaces multigroupes neutrons et gammas pour les calculs



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#### **Foreword**

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This document was prepared by Technical Committee ISO/TC 85, *Nuclear energy, nuclear technologies, and radiological protection*, Subcommittee SC 6, *Reactor technology*.

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# Group-averaged neutron and gamma-ray cross sections for radiation protection and shielding calculations for nuclear reactors

### 1 Scope

This document provides guidance in the preparation, verification, and validation of group-averaged neutron and gamma-ray cross sections for the energy range and materials of importance in radiation protection and shielding calculations for nuclear reactors<sup>1)</sup>, see also Annex A.

#### 2 Normative references

The following documents are referred to in the text in such a way that some or all of their content constitutes requirements of this document. For dated references, only the edition cited applies. For undated references, the latest edition of the referenced document (including any amendments) applies.

ISO 12749-5, Nuclear energy, nuclear technologies, and radiological protection — Vocabulary — Part 5: Nuclear reactors

#### 3 Terms and definitions

For the purposes of this document, the terms and definitions given in ISO 12749-5 and the following apply.

ISO and IEC maintain terminological databases for use in standardization at the following addresses:

- ISO Online browsing platform: available at <a href="https://www.iso.org/obp">https://www.iso.org/obp</a>
- IEC Electropedia: available at <a href="https://www.electropedia.org/">https://www.electropedia.org/</a>

#### 3.1

#### cross-section processing code

computer code that converts evaluated nuclear data in a specified format and procedure into a form that is appropriate for use in applications

Note 1 to entry: A cross-section processing code performs calculations such as resonance reconstruction, Doppler broadening, and multigroup averaging.

#### 3.2

#### ENDF/B

United States of America *evaluated nuclear data file* (3.3) prepared and reviewed by subject matter experts that is coordinated and maintained by CSEWG and NNDC at Brookhaven National Laboratory

#### 3 3

#### evaluated nuclear data file

nuclear reaction database stored using a specified format and procedure

EXAMPLE ENDF/ $B^{[2]}$ , JEFF $^{[3]}$ , and JENDL $^{[4]}$ .

<sup>1)</sup> This edition is based on ANSI/ANS-6.1.2-2013[1].